

Available online at www.sciencedirect.com



International Journal of HEAT and MASS TRANSFER

International Journal of Heat and Mass Transfer 47 (2004) 4415-4425

www.elsevier.com/locate/ijhmt

# Void fraction predictions in forced convective subcooled boiling of water between 10 and 18 MPa

J.M. Delhaye \*, F. Maugin, J.M. Ochterbeck

Department of Mechanical Engineering, Clemson University, Clemson, SC 29634, USA

Received 13 June 2003; received in revised form 25 March 2004

#### Abstract

Forced convective subcooled boiling is of interest for many industrial applications, especially as a possible heat transfer regime in the core of pressurized water reactors (PWR). Due to practical interest, R12 was used as a testing fluid to simulate water between 10 and 18 MPa. This article presents a study of the forced convective subcooled boiling of R12 based on previous models and new physical considerations. The axial void fraction profile is determined and compared to R12 experimental data obtained on the DEBORA loop at CEA/Grenoble.

The research yielded a void fraction model, shown to be valid for R12 as well as water, and based on a new correlation for the distribution parameter and on a continuous actual quality versus equilibrium quality function extended to the partially developed boiling region. Results showed good agreement with the DEBORA R12 experimental data as well as the experimental data obtained by Bartolomei et al. [Thermal Engng. 29(3) 132] in water at high pressure. © 2004 Elsevier Ltd. All rights reserved.

### 1. Introduction

Forced convective subcooled boiling is of interest for several applications in the power and process industry. In particular, subcooled boiling may be encountered in the core of the pressurized water reactors (PWRs) during startup, nominal or incidental conditions. This boiling flow configuration may lead to a boiling crisis and to burnout of the fuel cladding. Consequently, it is desirable to develop a model able to explain and predict behavior occurring in this type of boiling flow. An experiment performed with steam-water at high pressure to qualify these models would be extremely expensive. As a result, R12 has been used for simulating forced convective subcooled boiling in PWRs. In fact, this fluid can reproduce the same flow characteristics as water in PWR conditions but for a much lower pressure and a

0017-9310/\$ - see front matter C 2004 Elsevier Ltd. All rights reserved. doi:10.1016/j.ijheatmasstransfer.2004.05.018

much lower heat flux. The following similarity criteria were used (Appendix A):

- (a) Identical geometry.
- (b) Equivalent vapor/liquid density ratio to determine the corresponding range of pressure:

$$\left(\frac{\rho_{\rm g}}{\rho_{\rm f}}\right)_{\rm water} = \left(\frac{\rho_{\rm g}}{\rho_{\rm f}}\right)_{\rm R12} \tag{1}$$

where  $\rho_{\rm f}$  is the liquid density and  $\rho_{\rm g}$  is the vapor density.

(c) Same Weber number to calculate the corresponding range of mass flux *G*:

$$We \doteq \frac{G^2 D}{\sigma \rho_{\rm f}} \tag{2}$$

where D is the tube diameter and  $\sigma$  is the surface tension.

(d) Same Boiling number to calculate the corresponding range of heat flux  $\Phi$ :

$$Bo \triangleq \frac{\Phi}{Gh_{\rm fg}} \tag{3}$$

where  $h_{\rm fg}$  is the latent heat of vaporization.

<sup>&</sup>lt;sup>\*</sup>Corresponding author. Present address: CEA/Grenoble, DTN/SE2T, 38054 Grenoble Cedex 9, France. Fax: +33-4-38-78-31-96.

E-mail address: jean-marc.delhaye@cea.fr (J.M. Delhaye).

## Nomenclature

а	vapor volume at the wall per unit area of the wall	$v \\ \widetilde{V}_{ri}$	velocity weighted drift velocity (Eq. (B.4))
b	expansion ratio (Eq. (18))	We	Weber number (Eq. (2))
Bo	Boiling number (Eq. (3))	x	actual quality
$C_0$	distribution parameter (Eq. (B.3))	Xaa	equilibrium quality
$C_{01}$ , $C_{02}$	Hancox and Nicoll coefficients (Eqs. (24)	Z	coordinate along the channel axis
$c_p \\ D \\ Fr \\ g \\ G \\ h_\ell \\ h_{\rm fg} \\ j \\ j$	and (25)) specific heat tube diameter Froude number (Eq. (28)) acceleration due to gravity mass flux liquid specific enthalpy liquid specific enthalpy at saturation latent heat of vaporization local volumetric flux of the mixture (Eq.	Greek s α α β Φ μ ν ρ σ č	ymbols area void fraction local void fraction volumetric quality, cone angle heat flux viscosity kinematic viscosity density surface tension constant in the quality function
	(B.2))	5	1 5
T	······································	G 1 ·	
J	mixture volumetric flux	Subscrip	ots
J Jg	mixture volumetric flux gas volumetric flux	Subscrip c	critical
J J <sub>g</sub> k	mixture volumetric flux gas volumetric flux thermal conductivity tube length	Subscrip c eq	ots critical equilibrium coturated liquid
J J <sub>g</sub> k L	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37))	Subscrip c eq f	ots critical equilibrium saturated liquid saturated yapor
J J <sub>g</sub> k L L <sub>cap</sub>	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nubizndah coefficient (Eq. (27))	Subscrip c eq f g in	ots critical equilibrium saturated liquid saturated vapor inlat
J J <sub>g</sub> k L L <sub>cap</sub> n P	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27))	Subscrip c eq f g in i	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the
J Jg k L L <sub>cap</sub> n P P	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14))	Subscrip c eq f g in j	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture
J Jg k L Lcap n P Pe Pre	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14)) liquid Prandtl number (Eq. (9))	Subscrip c eq f g in j	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture liquid
$J$ $J_{g}$ $k$ $L$ $L_{cap}$ $n$ $P$ $Pe$ $Pr_{\ell}$ $a''$	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14)) liquid Prandtl number (Eq. (9)) critical heat flux	Subscrip c eq f g in j \ell l	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture liquid liquid only
$J \\ J_{g} \\ k \\ L \\ L_{cap} \\ n \\ P \\ Pe \\ Pr_{\ell} \\ q_{cr}'' \\ Re_{s}$	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14)) liquid Prandtl number (Eq. (9)) critical heat flux liquid Reynolds number (Eq. (8))	Subscrip c eq f g in j \ell ℓ ℓ0 ONB	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture liquid liquid only onset of nucleate boiling
$J \\ J_{g} \\ k \\ L \\ L_{cap} \\ n \\ P \\ Pe \\ Pr_{\ell} \\ q''_{cr} \\ Re_{\ell} \\ T$	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14)) liquid Prandtl number (Eq. (9)) critical heat flux liquid Reynolds number (Eq. (8)) temperature	Subscrip c eq f g in j \ell l0 ONB OSV	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture liquid liquid only onset of nucleate boiling onset of significant void
$J$ $J_{g}$ $k$ $L$ $L_{cap}$ $n$ $P$ $Pe$ $Pr_{\ell}$ $q''_{cr}$ $Re_{\ell}$ $T$ $T_{rat}$	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14)) liquid Prandtl number (Eq. (9)) critical heat flux liquid Reynolds number (Eq. (8)) temperature saturation temperature	Subscrip c eq f g in j l l ONB OSV sat	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture liquid liquid only onset of nucleate boiling onset of significant void saturation
$J \\ J_g \\ k \\ L \\ L_{cap} \\ n \\ P \\ Pe \\ Pr_{\ell} \\ q''_{cr} \\ Re_{\ell} \\ T \\ T_{sat} \\ T_{\ell in} $	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14)) liquid Prandtl number (Eq. (9)) critical heat flux liquid Reynolds number (Eq. (8)) temperature saturation temperature inlet liquid temperature	Subscrip c eq f g in j l l l ONB OSV sat v	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture liquid liquid only onset of nucleate boiling onset of significant void saturation vapor
$J$ $J_{g}$ $k$ $L$ $L_{cap}$ $n$ $P$ $Pe$ $Pr_{\ell}$ $q''_{cr}$ $Re_{\ell}$ $T$ $T_{sat}$ $T_{\ell,in}$ $\Delta T_{sat}$	mixture volumetric flux gas volumetric flux thermal conductivity tube length capillary length (Eq. (37)) Nabizadeh coefficient (Eq. (27)) pressure Péclet number (Eq. (14)) liquid Prandtl number (Eq. (9)) critical heat flux liquid Reynolds number (Eq. (8)) temperature saturation temperature inlet liquid temperature wall superheat (Eq. (16))	Subscrip c eq f g in j \ell l0 ONB OSV sat v w	ots critical equilibrium saturated liquid saturated vapor inlet relative to the local volumetric flux of the mixture liquid liquid only onset of nucleate boiling onset of significant void saturation vapor wall

(e) Same equilibrium inlet quality to find the corresponding range of inlet temperature:

$$x_{\rm eq,in} \stackrel{c}{=} \frac{h_{\ell,\rm in} - h_{\rm f}}{h_{\rm fg}} \tag{4}$$

where  $h_{\ell,\text{in}}$  is the liquid inlet specific enthalpy and  $h_{\text{f}}$  is the specific enthalpy of saturated liquid.

An experimental facility named DEBORA was built at CEA/Grenoble [1]. The test section (Fig. 1) is a 3.5 m long heated tube with an inner diameter of 19.2 mm. The control parameters are:

- the exit pressure,
- the mass velocity,
- the heat flux, and
- the inlet equilibrium quality,

which are imposed within the ranges corresponding to the PWR flow characteristics (Table 1). The following local, i.e. pointwise, quantities were measured by a twosensor optical probe:

- void fraction,
- interfacial area concentration,
- bubble center flux,
- bubble diameter,
- bubble Sauter mean diameter, and
- gas velocity.

The wall temperature was measured by means of four thermocouples placed along the tube external wall.

The vapor generation along a heated tube is represented in Fig. 2, where the most significant boundaries and regions are defined, and the longitudinal profiles of



Fig. 1. DEBORA experiment (adapted from [1]).



Fig. 2. Forced convective subcooled boiling.

void fraction, pressure drop and wall temperature are schematically represented. The onset of nucleate boiling (ONB) takes place where the first vapor bubbles appear from the nucleation sites. The onset of significant void (OSV) takes place where a significant increase in the void fraction occurs. Finally, forced convective boiling is Table 1

PWR flow characteristics and corresponding R12 flow characteristics of the DEBORA experiment

Parameters	Water	R12
Exit pressure (MPa)	10–18	1.4–3
Mass velocity (kg/m <sup>2</sup> s)	1000–5000	1000–5000
Heat flux (MW/m <sup>2</sup> )	0.6–6	0.05–0.65

said to be saturated when the liquid enthalpy reaches the liquid saturation enthalpy. The region between the ONB and the OSV is called the partially developed boiling region (PDB), while the region between the OSV and the saturation state is called the fully developed boiling region (FDB). The void fraction increases slightly from the ONB to the OSV and increases much faster in the FDB region. The pressure drop in this case is higher than the pressure drop in an equivalent single phase flow due to the presence of bubbles. As for the wall temperature, it increases linearly in single phase flow and remains almost constant in the subcooled boiling region.

In the present work, the area void fraction in forced convective subcooled boiling is calculated for imposed values of:

- the mass flux,
- the heat flux,
- the exit pressure, and
- the inlet equilibrium quality.

The void fraction is determined by a model based on the Zuber and Findlay [2] drift flux equation, a new relation between the actual and equilibrium quality and a new correlation for the distribution parameter. The results are then compared to the DEBORA experimental data as well as to the experimental data obtained by Bartolomei et al. [3] in water at high pressure.

## 2. Significant regime boundaries in subcooled boiling

The onset of nucleate boiling is defined as the point where the first bubbles appear from wall nucleation sites. An energy balance provides its location:

$$z_{\text{ONB}} = \frac{Gc_{p\ell}D}{4} \left[ \frac{(T_{\text{sat}} - T_{\ell,\text{in}}) + (\Delta T_{\text{sat}})_{\text{ONB}}}{\Phi} - \frac{1}{h_{\text{fg}}} \right]$$
(5)

where

$$\Delta T_{\rm sat} \stackrel{\circ}{=} T_{\rm w} - T_{\rm sat} \tag{6}$$

The convective heat transfer coefficient,  $h_{\ell 0}$ , in turbulent liquid flow is given by the Dittus–Boelter correlation:

$$\frac{h_{\ell 0}D}{k_{\ell}} = 0.023 R e_{\ell}^{0.8} P r_{\ell}^{0.4} \tag{7}$$

where  $k_{\ell}$  is the liquid conductivity,  $Re_{\ell}$  and  $Pr_{\ell}$  are the Reynolds number and the Prandtl number, respectively, and are defined by:

$$Re_{\ell} \triangleq \frac{GD}{\mu_{\ell}} \tag{8}$$

$$Pr_{\ell} \stackrel{\simeq}{=} \frac{\mu_{\ell} c_{p\ell}}{k_{\ell}} \tag{9}$$

In Eq. (7) the properties are evaluated at the film temperature  $[T_{\ell}(z_{\text{ONB}}) + T_{\text{w}}(z_{\text{ONB}})]/2$ , whereas the liquid specific heat is evaluated at the averaged temperature  $[T_{\ell}(z_{\text{ONB}}) + T_{\ell,\text{in}}]/2$ . The wall superheat,  $(\Delta T_{\text{sat}})_{\text{ONB}}$ , is estimated by the Frost and Dzakowic [4] correlation:

$$(\Delta T_{\rm sat})_{\rm ONB} = \left(\frac{8\sigma\Phi T_{\rm sat}}{k_{\rm f}h_{\rm fg}\rho_{\rm v}}\right)^{0.5} Pr_{\ell}$$
(10)

The properties in Eq. (10) are calculated at the saturation temperature. This correlation, which is known to be reliable for several fluids, is valid for water at pressures from 0.1 to 20 MPa and a heat flux of 0.15 MW/m<sup>2</sup>, and for R12 at pressures from 0.1 to 3 MPa and a heat flux of 0.15 MW/m<sup>2</sup>. There are no restrictions given about the range of mass flux.

The subcooling,  $\Delta T_{OSV}$ , at the point where a significant increase in the void fraction appears was determined by Saha and Zuber [5] that has been recognized to be very reliable:

• Pe < 70000

$$\Delta T_{\rm OSV} = 0.0022 \frac{\Phi D}{k_{\ell}} \tag{11}$$

• *Pe* > 70000

$$\Delta T_{\rm OSV} = 153.8 \frac{\Phi}{Gc_{p\ell}} \tag{12}$$

where

$$\Delta T_{\rm OSV} \stackrel{\circ}{=} T_{\rm sat} - T_{\ell}(z_{\rm OSV}) \tag{13}$$

$$Pe \doteq \frac{GDc_{p\ell}}{k_{\ell}} \tag{14}$$

A thermal balance then estimates the location of onset of significant void:

$$z_{\rm OSV} = z_{\rm ONB} + \{h_{\ell}[T_{\ell}(z_{\rm OSV})] - h_{\ell}[T_{\ell}(z_{\rm ONB})]\}\frac{GD}{4\Phi}$$
(15)

#### 3. Void fraction modeling

The void fraction is a parameter of interest because it is required to calculate the pressure drop and because it influences the neutronic response of the reactor core. Levy [6], Kroeger and Zuber [7], Saha and Zuber [5] and Lahey and Moody [8] proposed different methods to calculate the void fraction in subcooled boiling based on the Zuber and Findlay drift flux model (Appendix B). These models differ by the choice of (i) the relation between the actual and equilibrium quality, (ii) the distribution parameter, and (iii) the weighted drift velocity. Also, these models only calculate the void fraction in the fully developed boiling region and assume a zero void fraction in the partially developed boiling region. Levy [6] and Griffith et al. [9] proposed correlations for the void fraction at the OSV, but, to our knowledge, there are no models continuously calculating the void fraction from the ONB to the end of the subcooled boiling region. Manon [10] tested the above quoted models, compared them to the DEBORA experimental data and recommended the use of the Kroeger and Zuber model to calculate the void fraction for forced convective subcooled boiling of R12. However, the results show that this model is actually not very accurate and underestimates the void fraction. In the present work the possibility of improving the Lahey and Moody [8] model was investigated.

## 3.1. Lahey and Moody model

The objective of the Lahey and Moody [8] model is to estimate the area void fraction in the fully developed boiling region. The calculation is based on the Zuber and Findlay [2] model:

$$\alpha = \frac{x\rho_{\ell}G}{C_0[x\rho_{\ell} + (1-x)\rho_{\nu}]G + \widetilde{V}_{gj}\rho_{\ell}\rho_{\nu}}$$
(16)

where  $C_0$  is the distribution parameter given by Dix [11]:

$$C_0 = \beta \left[ 1 + \left( \frac{1}{\beta} - 1 \right)^b \right] \tag{17}$$

with

$$b \stackrel{\circ}{=} \left(\frac{\rho_{\rm v}}{\rho_{\ell}}\right)^{0.1} \tag{18}$$

where the volumetric quality,  $\beta$ , is given by:

$$\beta = \frac{x}{x + (1 - x)\rho_v/\rho_\ell} \tag{19}$$

The weighted drift velocity,  $\tilde{V}_{gj}$ , is given by:

$$\widetilde{V}_{gj} = 1.41 \left[ \frac{\sigma g(\rho_{\ell} - \rho_{\nu})}{\rho_{\ell}^2} \right]^{1/4}$$
(20)

The quality *x* is calculated as a function of the equilibrium quality  $x_{eq}$  according to the Levy [6] suggestion:

$$x(z) = x_{eq}(z) - x_{eq}(z_{OSV}) \exp\left[\frac{x_{eq}(z)}{x_{eq}(z_{OSV})} - 1\right]$$
(21)

where  $z_{OSV}$  is calculated with the Saha and Zuber model (Eqs. (11)–(15)) and where the equilibrium quality is defined as:

$$x_{\rm eq} \stackrel{\text{\tiny (22)}}{=} \frac{h_\ell - h_{\rm f}}{h_{\rm fg}}$$

As Lahey and Moody did not propose any correlation for the quality in the partially developed boiling (PDB) region between the ONB and the OSV, the void fraction in this region was assumed equal to 0 for all tests. Fig. 3 shows a comparison between the Lahey and Moody model and the DEBORA experimental data for R12. The comparison shows that for R12 the Lahey and Moody model overestimates the value of void fraction in the fully developed boiling region. Consequently, modifications to this model were desired to improve the agreement with the experimental data. Calculations showed that a change in the weighted drift velocity  $V_{gj}$  had very little influence on the calculated void fraction. As a result, modifications were focused on the distribution parameter  $C_0$ . In addition, a correlation is proposed for the quality in the PDB region to obtain a non-zero void fraction in this region.

#### 3.2. The distribution parameter $C_0$

Several models have been developed to estimate the value of the distribution parameter  $C_0$  in subcooled boiling. Some of these models are described below and the values are plotted over the range of parameters corresponding to the DEBORA experiment.

Dix [11] proposed to use Eqs. (17)–(19) for  $C_0$  as a function of quality (Fig. 4). The value of  $C_0$  depends on the pressure only and its variation is small between 1.4 and 2.6 MPa for a given void fraction.



Fig. 3. Comparison between Lahey and Moody model [8] and DEBORA data (R12).



Fig. 4. Distribution parameter in R12 calculated with Dix [11] model.



Fig. 5. Distribution parameter in R12 calculated with Hancox and Nicoll [12,13] model.

Hancox and Nicoll [12,13] proposed a value of  $C_0$  depending on the void fraction (Fig. 5):

$$C_0 = \left(\frac{1 - \exp(-C_{01}\alpha)}{1 - \exp(-C_{02})}\right)(1 + C_{02}) - C_{02}\alpha$$
(23)

with

$$C_{01} \cong 19$$
 (24)

$$C_{02} \stackrel{\circ}{=} 1.164 - 1.6534 \times 10^{-7}P + 7.5086 \times 10^{-15}P^2 \qquad (25)$$

where *P* is in Pa. The  $C_0$  calculated by the Hancox and Nicoll correlation [12,13] depends only on the pressure and the variations are small between 1.4 and 3 MPa. However, the values are too high, as Hoffman and Wong [14] already noticed.

Saha and Zuber [5] assumed that  $C_0$  was a constant equal to 1.13. However,  $C_0$  is not a constant for sub-



Fig. 6. Distribution parameter in R12 calculated with Nabizadeh [15] model.

cooled boiling since the void fraction and the velocity profiles are not fully developed.

Nabizadeh [15] proposed a correlation for  $C_0$  involving the pressure, the mass flux and the quality (Fig. 6):

$$C_{0} = \left(1 + \frac{1 - x}{x} \frac{\rho_{v}}{\rho_{\ell}}\right)^{-1} \left[1 + \frac{1}{n} Fr^{-0.1} \left(\frac{\rho_{v}}{\rho_{\ell}}\right)^{n} \left(\frac{1 - x}{x}\right)^{11n/9}\right]$$
(26)

where

$$n \stackrel{\text{\tiny}}{=} \sqrt{0.6 \frac{\rho_{\ell} - \rho_{\rm v}}{\rho_{\ell}}} \tag{27}$$

$$Fr \stackrel{\circ}{=} \frac{G^2}{g D \rho_\ell^2} \tag{28}$$

The Lahey and Moody [8] model was tested against the DEBORA data with the three distribution parameters models described above. The location of the OSV was calculated with Saha and Zuber model (Eqs. (11)– (15)) and the void fraction at the OSV was assumed equal to 0 for these tests. The results are shown in Fig. 7 along with the DEBORA experimental data. The best results were obtained with Hancox and Nicoll model but a deviation still occurs at 1.46 MPa.

To account for the mass flux and heat flux effects, we assumed that the distribution parameter was a function of the Reynolds number and of the Boiling number. A fit on the DEBORA void fraction data led to the following correlation:

$$C_0 = 0.087 \left(\frac{P}{P_c}\right)^{0.8} \frac{Re_{\ell}^{0.1}}{Bo^{0.28}} \left[1 - \exp\left(-\frac{\alpha}{0.05}\right)\right]$$
(29)

where *Bo* and  $Re_t$  are the Boiling number and the liquid Reynolds number defined by Eqs. (3) and (8), respectively. The variations of the distribution parameter are



Fig. 7. Comparison between the Lahey and Moody [8] model and the DEBORA data for three different correlations of the distribution parameter.



Fig. 8. Distribution parameter calculated with Eq. (29).

represented in Fig. 8. The above  $C_0$  correlation is valid for the following DEBORA parameter ranges:

$$3.4 \times 10^5 < Re_{\ell} < 8.6 \times 10^5$$
  
 $2.3 \times 10^{-4} < Bo < 7.2 \times 10^{-4}$ 

#### 3.3. Quality in the partial developed boiling region

The Levy model recommended by Lahey and Moody to calculate the quality assumes that the quality is 0 in the PDB region. However, such an assumption is not substantiated by the experimental data.

Facing the inconvenience of the Lahey and Moody model, Manon [10] modified the Levy model to obtain a non-zero value at the OSV:

$$x = x_{\rm eq} + [x_{\rm OSV} - x_{\rm eq}(z_{\rm OSV})] \exp\left[\frac{x_{\rm eq}}{x_{\rm eq}(z_{\rm OSV})} - 1\right]$$
(30)

where  $x_{OSV}$  is the quality at the OSV calculated by:

$$x_{\rm OSV} = \frac{1}{2} \frac{\rho_{\rm v}}{\rho_{\ell}} \alpha_{\rm OSV} \tag{31}$$

and  $\alpha_{OSV}$  is the void fraction calculated by the Griffith et al. [9] model detailed and modified in Section 3.5. Manon [10] also proposed to use a linear model for the void fraction between the ONB and the OSV.

In order to be more realistic, we propose to represent the quality in the PDB region by a hyperbolic tangent function having a zero derivative at the ONB:

$$x = 0.01\xi \left\{ x_{\text{eq}} - x_{\text{eq}}(z_{\text{ONB}}) \left\{ \tanh\left[\frac{x_{\text{eq}}}{x_{\text{eq}}(z_{\text{ONB}})} - 1\right] + 1 \right\} \right\}$$
(32)

where  $z_{\text{ONB}}$  is the location of the ONB calculated by an energy balance (Eq. (5)) and  $\xi$  is a constant to be adjusted in order to allow continuity of the quality between the PDB region and the FDB region. The procedure to adjust  $\xi$  is the following:

1. The equilibrium quality for which the derivative of Eqs. (30) and (32) are the same is found by iteration. All tests conducted show that this equilibrium quality is very close to the equilibrium quality at the OSV.



Fig. 9. Quality profile calculated with Eqs. (30) and (32).

2. Then the constant  $\xi$  is adjusted so that Eqs. (30) and (32) give the same value of x where their derivatives are equal.

Fig. 9 shows a typical graph of the actual quality, x, versus the equilibrium quality  $x_{eq}$ .

## 3.4. Void fraction at the OSV

To evaluate  $x_{OSV}$  it is necessary to determine the void fraction  $\alpha_{OSV}$  at the OSV. We suggest to use the Griffith et al. [9] model developed from experimental data obtained for water in the ranges given in Table 2. This model estimates the vapor volume at the wall per unit area of the wall:

$$a = \frac{\Phi k_{\ell} P r_{\ell}}{1.07 h_{\ell_0}^2 [T_{\text{sat}} - T_{\ell}(z_{\text{OSV}})]}$$
(33)

where the liquid temperature  $T_{\ell}(z_{\text{OSV}})$  is estimated by an energy balance between the inlet and  $z_{\text{OSV}}$ , assuming that the flow is single phase:

$$T_{\ell}(z) = T_{\ell,\text{in}} + \frac{4\Phi_z}{Gc_{\rho\ell}D}$$
(34)

Table 2

Ranges of	experimental	data u	sed by	Griffith	et al.	[9]
-----------	--------------	--------	--------	----------	--------	-----

Parameter	Water	Corresponding ranges in R12
Pressure (MPa)	3.4, 6.9 and 10.3	0.6, 1.2 and 1.8
Mass flux (kg/m <sup>2</sup> s)	80-400	80-400
Heat flux (MW/m <sup>2</sup> )	1.6-8.5	0.16-0.93

#### Table 3

Void fraction at the onset of significant void: comparison between Bartolomei et al. experimental data and Griffith et al. model for void fraction at the OSV

Parameters	Experimental data	Griffith et al. model
P = 147  bar $G = 2014 \text{ kg/m}^2 \text{ s}$ $\Phi = 1720 \text{ kW/m}^2$	$0.02 \pm 0.02$	0.02
P = 68.9  bar $G = 988 \text{ kg/m}^2 \text{ s}$ $\Phi = 1130 \text{ kW/m}^2$	$0.02\pm0.02$	0.03
P = 108.1  bar $G = 966 \text{ kg/m}^2 \text{ s}$ $\Phi = 1130 \text{ kW/m}^2$	$0.02 \pm 0.02$	0.03

Table 4 Void fraction at the onset of significant void: comparison between DEBORA experimental data and the modified Griffith et al. model

Parameters	Experimental data	Griffith et al. model	Modified Griffith et al. model
P = 1.46  MPa $G = 5020 \text{ kg/m}^2 \text{ s}$ $\Phi = 135 \text{ kW/m}^2$	0.01±0.02	0.04	0.01
P = 1.46  MPa $G = 5020 \text{ kg/m}^2 \text{ s}$ $\Phi = 161 \text{ kW/m}^2$	$0.01 \pm 0.02$	0.04	0.01
P = 1.45 MPa G = 2010 kg/m <sup>2</sup> s $\Phi = 76.2$ kW/m <sup>2</sup>	$0.02 \pm 0.02$	0.06	0.018
P = 1.46 MPa G = 2020 kg/m <sup>2</sup> s $\Phi = 76.2$ kW/m <sup>2</sup>	$0.02 \pm 0.02$	0.06	0.018
P = 2.62  MPa $G = 1990 \text{ kg/m}^2 \text{ s}$ $\Phi = 73.8 \text{ kW/m}^2$	$0.01 \pm 0.02$	0.05	0.01
P = 2.6  MPa $G = 3000 \text{ kg/m}^2 \text{ s}$ $\Phi = 179 \text{ kW/m}^2$	$0.01 \pm 0.02$	0.04	0.008
P = 2.6  MPa $G = 3000 \text{ kg/m}^2 \text{ s}$ $\Phi = 138 \text{ kW/m}^2$	$0.01 \pm 0.02$	0.04	0.008
P = 2.6  MPa $G = 3000 \text{ kg/m}^2 \text{ s}$ $\Phi = 188 \text{ kW/m}^2$	$0.01 \pm 0.02$	0.04	0.008

where the liquid specific heat,  $c_{\rho\ell}$ , is estimated at the temperature  $[T_{\ell,\text{in}} + T_{\ell}(z_{\text{OSV}})]$ . The single phase heat transfer coefficient,  $h_{\ell 0}$ , is determined by the Dittus–Boelter correlation (Eq. (7)). All the liquid properties in Eq. (33) are taken at the OSV temperature. The empirical constant 1.07 was adjusted by the authors on their experimental data. Griffith et al. considered that the partially developed boiling region is governed by the correlation given in Eq. (33) as long as *a* is lower than 0.1 mm. The void fraction at the OSV,  $\alpha_{\text{OSV}}$ , is finally given by:

$$\alpha_{\rm OSV} = \frac{4a}{D} \tag{35}$$

The Griffith et al. model is compared in Table 3 to the experimental data obtained for *water* by Bartolomei et al. [3]. Considering the accuracy of the measurements, the Griffith et al. model gives a good estimate of the void fraction at the OSV. The same model was used with R12 and the results compared to the DEBORA data bank. Table 4 shows that the Griffith et al. model overestimates the value of void fraction at the OSV in R12, which is not surprising since the model was developed for water.

In order to adapt the Griffith et al. model to R12 and to any other fluid, the effect of the surface tension of the fluid was included by introducing the Laplace length in the expression of *a*:

$$a = 7.5 \frac{\Phi k_{\ell} P r_{\ell}}{h_{\ell 0}^2 (T_{\text{sat}} - T_{\ell})} \frac{L_{\text{cap}}}{D}$$

$$(36)$$

where  $L_{cap}$  is the capillary length defined by:

$$L_{\rm cap} \stackrel{\circ}{=} \sqrt{\frac{\sigma}{g(\rho_{\ell} - \rho_{\rm v})}} \tag{37}$$

The modified model gives the same results as the original Griffith et al. model for water and is compared to DEBORA experimental data in Table 4. The comparison shows that the modified Griffith et al. model (Eqs. (35)–(37)) gives a good estimation of  $\alpha_{OSV}$ .

## 3.5. Final model

From the results and discussion above, a new model predicting the evolution of void fraction in subcooled boiling has been developed and is summarized below:

- onset of nucleate boiling (Frost and Dzakowic): Eqs. (5)–(10),
- onset of significant void (Saha and Zuber): Eqs. (11)–(15),
- Zuber and Findlay model: Eq. (16),
- weighted drift velocity: Eq. (20),
- distribution parameter: Eqs. (3), (8) and (29),
- actual quality: Eqs. (30)–(32),

• void fraction at the onset of significant void: Eqs. (34)–(37).

Fig. 10 shows some comparisons between the results obtained with this new model and the DEBORA experimental data. The prediction of the void fraction in the PDB region as well as in the FDB region in forced convective boiling of R12 has been remarkably improved with respect to the Lahey and Moody model (Fig. 3). Most of our results are within the range of accuracy of the DEBORA experiment, which is  $\pm 0.02$ for the void fraction. In addition, Fig. 11 shows a comparison with Bartolomei et al. [3] experimental data. The model developed also gives a good estimation of the void fraction in forced convective subcooled boiling of water, which further substantiates its physical soundness. Finally, Fig. 12 summarizes all the results obtained for water and R12. It shows that the suggested  $C_0$  correlation (29) that was fitted on the DEBORA data can be extended to the Bartolomei et al. data. Consequently,



Fig. 10. Comparison between the results of the proposed model and the DEBORA experimental data (R12).



Fig. 11. Comparison between the results of the proposed model and the Bartolomei et al. [3] experimental data (water).



Fig. 12. The proposed model compared to DEBORA (R12) and Bartolomei et al. [3] (water) experimental data.

its validity actually covers the following values of the Reynolds and Boiling numbers:

$$1.4 \times 10^5 < Re_\ell < 8.6 \times 10^5$$
  
 $2.3 \times 10^{-4} < Bo < 9.4 \times 10^{-4}$ 

## 4. Conclusions

Existing models were adapted and improved in order to calculate the void fraction in forced convective subcooled boiling of R12. A correlation was developed for the distribution parameter used in the Lahey and Moody [8] model. A new equation giving the actual quality versus the equilibrium quality was proposed. The Griffith et al. [9] model giving the void fraction at the onset of significant void was modified and is now valid for water *and* R12. Fig. 12 shows that the proposed void fraction model based on these modified submodels predicts very well the void fraction in subcooled boiling of water *and* R12 within the parameters ranges given in Table 1.

#### Acknowledgements

This study has been sponsored by EDF (Electricité de France) and CEA (Commissariat à l'Energie Atomique). The authors are fully indebted to Dr. Jacques Garnier, CEA/Grenoble, for his numerous and valuable comments.

## Appendix A. Similarity criteria used on Debora

Assume that we have the following functional relationship for the critical heat flux  $q'_{cr}$ :

 $q_{\mathrm{cr}}'' = F(\rho_{\mathrm{g}}, \rho_{\mathrm{f}}, G, D, \sigma, h_{\mathrm{fg}}, x_{\mathrm{eq,in}}, L)$ 

Following Kay and Nedderman [16, pp. 134 et seq.] we can introduce the enthalpy, or thermal energy, as a dimension. A straightforward calculation based on dimensional analysis leads to the five following five dimensionless groups:

- The Boiling number:  $Bo \cong \frac{\Phi}{Gh_{fg}}$
- The Weber number:  $We \triangleq \frac{G^2 D}{\sigma \rho_f}$ .
- The density ratio:  $\rho_g/\rho_f$ .
- The aspect ratio: L/D.
- The inlet equilibrium quality:  $x_{eq,in}$ .

These are the similarity criteria used in Debora.

### Appendix B. The Zuber and Findlay model

Zuber and Findlay [17] defined a local drift velocity,  $v_{gi}$ , by the following relation:

$$v_{\rm gj} = v_{\rm g} - j = (1 - \alpha_{\rm g})(v_{\rm g} - v_{\rm f})$$
 (B.1)

where  $\alpha_g$ ,  $v_g$  and  $v_f$  are the local void fraction, the local gas velocity and the local liquid velocity, respectively, and where *j* is the local volumetric flux of the mixture defined as:

$$j \stackrel{\circ}{=} \alpha_{\rm g} v_{\rm g} + (1 - \alpha_{\rm g}) v_{\rm f} \tag{B.2}$$

This relation can also be written as:

 $\alpha_{\rm g} v_{\rm gj} = \alpha_{\rm g} v_{\rm g} - \alpha_{\rm g} j$ 

The Zuber and Findlay equation is obtained by taking the average of the previous relation over the pipe cross section and reads:

$$lpha = rac{J_{
m g}}{C_0 J + \widetilde{V}_{
m gj}}$$

It gives the area void fraction,  $\alpha$ , as a function of the gas and mixture volumetric fluxes  $J_g$  and J if the two following quantities are known:

• The distribution parameter,  $C_0$ , defined as:

$$C_0 \stackrel{\scriptscriptstyle c}{=} \frac{\langle \alpha_{\rm g} j \rangle}{\langle \alpha_{\rm g} \rangle \langle j \rangle} \tag{B.3}$$

which depends strongly on the radial profile of the local void fraction, hence its importance in subcooled boiling.

• The weighted drift velocity,  $\tilde{V}_{gj}$ ,

$$\widetilde{V}_{\rm gj} = \frac{\langle \alpha_{\rm g} v_{\rm gj} \rangle}{\langle \alpha_{\rm g} \rangle} \tag{B.4}$$

which also depends strongly on the radial profile of the local void fraction.

The distribution parameter and the weighted drift velocity are always determined from experimental database. A direct determination from the definitions would require instrumentation techniques to measure the gas and liquid velocities accurately. Unfortunately these techniques are not yet available.

## References

 J. Garnier, E. Manon, G. Cubizolles, Local measurements on flow boiling of refrigerant 12 in a vertical tube, Multiphase Sci. Technol. 13 (2001) 1–111.

- [2] N. Zuber, J.A. Findlay, Average volumetric concentration in two phase flow systems, J. Heat Transfer 87 (1965) 453–467.
- [3] G.G. Bartolomei, V.G. Brantov, Yu.S. Molochnikov, Yu.V. Kharitonov, V.A. Solodkii, G.N. Batashova, V.N. Mikhailov, An experimental investigation of true volumetric vapour content with subcooled boiling in tubes, Thermal Engng. 29 (3) (1982) 132–135.
- [4] W. Frost, G.S. Dzakowic, An extension of the method of predictive incipient boiling on commercially finished surfaces, in: ASME-AIChE Heat Transfer Conference, Seattle, 1967.
- [5] P. Saha, N. Zuber, Point of net vapor generation and vapor void fraction in subcooled boiling, in: Fifth Int. Heat Transfer Conf., Tokyo, Paper B4.7, 1974.
- [6] S. Levy, Forced convection subcooled boiling-prediction of vapor volumetric fraction, Int. J. Heat Mass Transfer 19 (1) (1967) 99–113.
- [7] P.G. Kroeger, N. Zuber, An analysis of the effects of various parameters on the average void fractions in subcooled boiling, Int. J. Heat Mass Transfer 11 (1968) 211–233.
- [8] R.T. Lahey Jr., F.J. Moody, The Thermal Hydraulics of a Boiling Water Nuclear Reactor, American Nuclear Society, LaGrange Park, IL, 1977.
- [9] P. Griffith, J.A. Clark, W.M. Roshenow, Void volumes in subcooled boiling systems, ASME Paper 58-HT-19, ASME, New York, 1958.
- [10] E. Manon, Contribution à l'analyse et à la modélisation locale des écoulements bouillants sous-saturés dans les conditions des réacteurs a eau sous pression, Thèse de doctorat, Ecole Centrale de Paris, 2000.
- [11] G.E. Dix, Vapor Void Fractions for Forced Convection with Subcooled Boiling at Low Flow Rate, Ph.D. Thesis, University of California, 1971.
- [12] W.T. Hancox, W.B. Nicoll, Forced convection boiling; prediction of axial void distributions, Submitted for publication in the Can. J. Chem. Engng., quoted in Hancox and Nicoll [13].
- [13] W.T. Hancox, W.B. Nicoll, A general technique for the prediction of void distributions in non-steady two-phase forced convection, Int. J. Heat Mass Transfer 14 (3) (1971) 1377–1394.
- [14] M.A. Hoffman, C.F. Wong, Prediction of pressure drops in forced convection subcooled boiling water flows, Int. J. Heat Mass Transfer 35 (12) (1992) 3291–3299.
- [15] H. Nabizadeh, Ubertragungsgesetze für den Dampfvolumenanteil zwischen Freon und Wasser, Kolloquium über Aehnlichkeitsgesetze in Zweiphasenströmungen, Institut für Verfahrenstechnik der T.U. Hannover.
- [16] J.M. Kay, R.M. Nedderman, An Introduction to Fluid Mechanics and Heat Transfer, third ed., Cambridge University Press, Cambridge, 1974, p. 134 et seq.
- [17] N. Zuber, J.A. Findlay, J. Heat Transfer (1965) 453-458.